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Gary J. Taylor Vice President **Nuclear Operations**

May 20, 1997 RC-97-0107

Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555

Gentlemen:

VIRGIL C. SUMMER NUCLEAR STATION (VCSNS) Subject:

DOCKET NO. 50/395

OPERATING LICENSE NO. NPF-12

Licensee Event Report, (LER 970001)

Attached is Licensee Event Report No. 970001 for the Virgil C. Summer Nuclear Station . This report is submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Should you have any questions, please call Mr. Jeffrey Pease at (803) 345-4124.

Very truly yours,

JWP/GJT/nkk

Attachment

J.L.Skolds

W.F.Conway

R.R.Mahan (w/o attachment)

R.J.White

L.A.Reyes

A.R. Johnson

R.B.Clary

S.F. Fipps

A.R.Koon

G.E. Williams

T.L.Matlosz

K.R.Jackson

D.L.Abstance

NPRDS Coordinator

NRC Resident Inspector

IE 22

J.B.Knotts, Jr.

INPO Records Center

Marsh & McLennan

NSRC

RTS (LER 970001, CER 970353)

File (818.05, 818.07)

DMS (RC-97-0107)

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NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB NO. 3150-0104 (4-95) **EXPIRES 04/30/98** ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-8 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF LICENSEE EVENT REPORT (LER) MANAGEMENT AND BUDGET, WASHINGTON, DC 20505-0001, AND TO MANAGEMENT AND BUDGET, WASHINGTON, DC 20503. FACILITY NAME (1) DOCKET NUMBER (2) 05000395 1 OF 3 Virgil C.Summer Nuclear Station TITLE (4) Manual Reactor Trip EVENT DATE (5) LER NUMBER (6) REPORT DATE (7) OTHER FACILITIES INVOLVED (8) DAY 05000 FACILITY NAME DOCKET NUMBER 0 4 2 2 2 0 97 0 05 97 001 0 05000 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11) OPERATING MODE (9) 20.2201(b) 20.2203(a)(2)(v) 50.73(a)(2)(i) 50.73(a)(2)(viii) POWER 20.2203(a)(1) 20.2203(a)(3)(i) 50.73(a)(2)(ii) 50.73(a)(2)(x) LEVEL (10) 100 20.2203(a)(2)(i) 20.2203(a)(3)(ii)0 50.73(a)(2)(iii) 73 71 20.2203(a)(2)(ii) OTHER 20.2203(a)(4) 50.73(a)(2)(iv) specify in Abstract below 50.36(c)(1) 50.73(a)(2)(v) 20.2203(a)(2)(iii) or in NRC FORM 366A 50.73(a)(2)(vii) 20.2203(a)(2)(iv) 50.36(c)(2) LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER (Include Area Code) NAME April R.Rice, Manager, Nuclear Licensing & Operating Experience (803) 345-4232 COMPLETE ONE UNE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (B) CAUSET SYSTEM I COMPONENT I MANUFACTURER I REPORTABLE CAUSETSYSTEMICOMPONENTIMANUFACTURER REPORTABLE TO NPRDS TO NPRDS

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (15)

SUPPLEMENTAL REPORT EXPECTED (14)

On April 22, 1997, plant personnel conservatively initiated a manual reactor trip in response to a leak on the Main Turbine Electro-Hydraulic Control (EHC) system. It was subsequently determined that an "O" ring on the shutdown servo to Combined Intercept Valve (CIV) #1 had failed. Although Feedwater Isolation was achieved, one Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design. Repairs were accomplished and the plant restarted.

NO

D (Seeman)

SURMISSION

DATE

MONTH

DAY

YEAR

EB

TG

PIPEXXA

(If yes, complete EXPECTED SUBMISSION DATE).

NRC FORM 366A

(5-92)

U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB NO. 3150-0104

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

EXPIRES 5/31/95

FACILITY NAME (1)	DOCKET NUMBER	LER NUMBER (6)			PAGE (3)
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT IDENTIFICATION

Westinghouse - Pressurized Water Reactor

EQUIPMENT IDENTIFICATION

Hydraulic "O" ring in Electro-Hydraulic Control System EIIS System Code - TG

IDENTIFICATION OF EVENT

Manual Reactor Trip

EVENT DATE

April 22, 1997

REPORT DATE

May 20, 1997

This report was initiated by CER 970353

CONDITIONS PRIOR TO EVENT

MODE 1 - 100% Reactor Power

DESCRIPTION OF EVENT

On April 22, 1997, plant personnel conservatively initiated a manual reactor trip in response to a leak on the Main Turbine Electro-Hydraulic Control (EHC) system. It was subsequently determined that an "O" ring on the shutdown servo to Combined Intercept Valve (CIV) #1 had failed. Although Feedwater Isolation was achieved, one Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design. Repairs were accomplished and the plant restarted.

NRC FORM 366A

(5-92)

U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB NO. 3150-0104

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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V. C. Summer Nuclear Station	05000395	9 7	001	0 0	3 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CAUSE OF EVENT

The cause of the manual reactor trip is attributed to conservative operator action in anticipation of a Turbine trip due to the mechanical failure of a Hydraulic "O" ring in the Main Turbine Electro-Hydraulic Control (EHC) system.

ANALYSIS OF EVENT

One Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design.

IMMEDIATE CORRECTIVE ACTIONS:

The hydraulic "O" ring in the EHC system was replaced. Mechanical grooming of the FRV was accomplished. The outage package was reviewed for additional items which could be worked to enhance plant performance. These items were identified and worked prior to plant restart.

ADDITIONAL CORRECTIVE ACTIONS:

SCE&G is evaluating the failure of the hydraulic "O" ring to determine whether additional corrective actions will be required for this non-safety related component. This determination will be completed prior to the end of refueling outage 10, scheduled to begin in October 1997.

PRIOR OCCURRENCES:

None